July 22, 1985
NRC/TMI-85-053

MEMORANDUM FOR: Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Bernard J. Snyder, Program Director
TMI Program Office

FROM: William D. Travers, Deputy Program Director
TMI Program Office

SUBJECT: NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT FOR JULY 15, 1985 - JULY 21, 1985

1. PLANT STATUS

- The facility remains in long term cold shutdown with the Reactor Coolant System (RCS) vented to the reactor building atmosphere and the reactor vessel head and plenum assembly removed.
- The reactor vessel plenum has been removed from the reactor vessel and placed on its storage stand in the deep end of the fuel transfer canal. A dam has been installed between the deep and shallow ends of the fuel transfer canal. The deep end is filled with water to a depth of about 20 feet (about 5 feet above the top of the plenum).
- The modified internals indexing fixture is installed on the reactor vessel flange and is flooded to elevation 327 feet 6 inches (15½ feet above the top of the core region).
- Calculated reactor decay heat is less than 12 kilowatts.
- RCS cooling is by natural heat loss to the reactor building ambient atmosphere. Incore thermocouple readings range from 71°F to 92°F with an average of 81°F. Average cold leg temperature is 60°F.
- The average reactor building temperature is 60°F. The reactor building airborne activity is 2.3 E-7 uCi/cc tritium, and 1.5 E-10 uCi/cc particulate, predominantly cesium 137.

2. WASTE MANAGEMENT

- The Submerged Demineralizer System (SDS) commenced processing batch S123 consisting of 25,745 gallons from the "C" reactor coolant bleed tank. EPICOR II completed processing batch 260 consisting of 5,324 gallons from SDS tank 1B.
- Total volume processed through SDS to date is 2,939,092 gallons, and the total volume processed through EPICOR II is 2,505,311 gallons.

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3. DOSE REDUCTION/DECONTAMINATION ACTIVITIES

- Decontamination activities are continuing on the 281' level of the Auxiliary Building.
- Average general area radiation dose rate is 36 mrem per hour on the 347' level of the reactor building and is 160 mrem per hour on the 305' level of the reactor building.

4. ENVIRONMENTAL MONITORING

- EPA sample analysis results show TMI site liquid effluents to be in accordance with regulatory limits, NRC requirements, and the City of Lancaster Agreement.
- TMI water samples taken by the US Environmental Protection Agency at the plant discharge to the river consisted of seven daily composite samples taken from June 29, to July 6, 1985. Gamma scans detected no reactor related radioactivity.
- The Lancaster water samples taken at the water works intake and analyzed by the US Environmental Protection Agency consisted of seven day composite samples taken from June 30, to July 6, 1985 and July 7 to July 13, 1985. A gamma scan detected no reactor related radioactivity.
- The NRC outdoor airborne particulate sampler at the TMI Site collected a sample between July 11, and July 18, 1985. No reactor related radioactivity was detected. Analysis showed I-131 and Cs-137 concentrations to be less than the lower limits of detectability.

5. REACTOR BUILDING ACTIVITIES

- Additional characterization of the reactor vessel lower head was performed during the week of July 14, 1985. The characterization included: water sampling in the lower head and annulus regions to compare chemical properties for homogeniety, additional video inspections in two lower head quadrants which had not been examined previously, and sampling of debris in the lower head. The scheduled activities were completed in one of the two quadrants and the analysis of results is in progress. Solid debris samples will be sent to the INEL (Idaho National Engineering Laboratory) for study. Inspection of the second quadrant is scheduled to start Monday, July 22, 1985.

  Preliminary data indicates that boron concentrations, as well as other chemical and physical water properties in the lower head area, are consistent with those elsewhere in the vessel and that the rubble bed in the newly viewed area generally consists of smaller particles than previously seen in other areas.

Next week, reactor defueling preparations will dominate the scheduled activities in the reactor building. The defueling platform will be assembled and installed above the internals indexing fixture in conjunction with the trolley on the debris canister transfer bridge. The starting date for initial defueling has been revised from September to October 1985.

6. AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES

- Installation of the DWCS continued. Partial DWCS turnover for processing RCS during early defueling is scheduled to be completed in late August.

7. NRC EVALUATIONS IN PROGRESS

- Defueling Water Cleanup System Technical Evaluation (including Revision 6)
- Technical Specification Change Requests numbers 46, 48, 49, and 50
- Recovery Operations Plan Change numbers 27, 29, 31, and 32
- Fuel Canister Technical Evaluation
- Fuel Handling Senior Reactor Operator Training Program
- Defueling Safety Evaluation
- Application for seismic exemption
- The NRC Vendor Programs Branch performed an inspection at the Nuclear Energy Services (NES) facility in Greensboro, North Carolina, where the fuel storage canisters, fuel storage racks, and fuel canister transfer shields are being fabricated. The inspection examined construction activities and included a review of the implementation of the quality assurance program at NES. The inspection findings are undergoing NRC management review and the final inspection report will be used in a few weeks.

8. PROJECTED SCHEDULE OF FUTURE EVENTS

- Start of Defueling: October 1985

9. PUBLIC MEETING

- On July 18, 1985, the Advisory Panel for the Decontamination of Three Mile Island Unit 2 held a public meeting at the Public Safety Building in Lancaster, Pa. The Panel received a status report on the cleanup by GPU Nuclear Corporation, including an announcement that the scheduled commencement of defueling has been changed from September 1985 to October 1985 due to equipment procurement and delivery delays.

  Dr. William Travers, Deputy Program Director, TMIPO, NRR, provided a status report on NRC activities required to be completed prior to the commencement of defueling. Dr. William Kirk, Director, TMI

Field Station, EPA, described the upgraded aquatic radiological monitoring program in the vicinity of the TMI site; and Mr. Frank Congel, Chief, Radiological Assessment Branch, NRC, summarized the results of a staff review of health effects studies conducted in the vicinity of TMI-2 as a result of the March 1979 accident. The next meeting of the Advisory Panel is scheduled for September 1985 at a location in Annapolis, Maryland, to be identified later.

ORIGINAL SIGNED BY:
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